

are contained in a Safety Evaluation dated February 14, 1995.

Attorney for licensee: Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60690.

Local Public Document Room location: Public Library of Illinois Valley Community College, Rural Route No. 1, Oglesby, Illinois 61348.

NRC Project Director: Robert A. Capra.

Dated at Rockville, Maryland, this 21st day of February 1995.

For the Nuclear Regulatory Commission.

John N. Hannon,

Acting Director, Division of Reactor Projects—III/IV, Office of Nuclear Reactor Regulation.

[FR Doc. 95-4870 Filed 2-28-95; 8:45 am]

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[Docket Nos. 50-245, 50-336, 50-423]

Northeast Utilities; Issuance of Director's Decision Under 10 CFR 2.206

[Millstone Nuclear Power Station]
[License Nos. DPR-21, DPR-65, NPF-49]

Notice is hereby given that the Acting Director, Office of Enforcement, has issued a decision concerning the Petitions filed by Ms. Carmela V. Marien and Ms. Marianne W. Nericcio on August 21, 1993. The Petitions requested that the NRC initiate an investigation and accelerated enforcement action against Northeast Utilities (Licensee) for willful violation of the employee protection provisions of 10 CFR 50.7.

After due consideration of Petitioner's assertions, the Acting Director, Office of Enforcement, has denied the Petitions. The reasons for the denial are explained in the "Director's Decision under 10 CFR 2.206" (DD-95-04) which is available for public inspection in the Commission's Public Document Room at 2120 L Street NW., Washington, DC 20555.

A copy of this decision will be filed with the Secretary for the Commission's review in accordance with 10 CFR 2.206. As provided by this regulation, the decision will constitute the final action of the Commission 25 days after the date of issuance of the decision unless the Commission on its own motion institutes a review of the decision within that time.

Dated at Rockville, Maryland this 22nd day of February 1994.

For the Nuclear Regulatory Commission.

Joseph R. Gray,

Acting Director, Office of Enforcement.

[FR Doc. 95-4978 Filed 2-28-95; 8:45 am]

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[Docket No. 50-219]

GPU Nuclear Corporation; Notice of Issuance of Amendment to Facility Operating License

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 177 to Facility Operating License No. NPF-16 issued to GPU Nuclear Corporation (the licensee), which revised the Technical Specifications for operation of the Oyster Creek Nuclear Generating Station located in Ocean County, New Jersey. The amendment is effective as of the date of issuance, to be implemented within 60 days of issuance.

The amendment revises Technical Specification 2.3.D to change the setpoints "Reactor High Pressure, Relief Valve Initiation" by increasing the setpoint value by 15 psig for each of the Electromatic Relief Valves in the Automatic Depressurization System.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the **Federal Register** on July 5, 1994 (59 FR 34453). No request for a hearing or petition for leave to intervene was filed following this notice.

The Commission has prepared an Environmental Assessment related to the action and has determined not to prepare an environmental impact statement. Based upon the environmental assessment, the Commission has concluded that the issuance of the amendment will not have a significant effect on the quality of the human environment (60 FR 9056).

For further details with respects to the action see (1) The application for amendment dated June 15, 1994, as supplemented September 23, and November 23, 1994, (2) Amendment No. 177 to License No. DPR-16, (3) the Commission's related Safety Evaluation, and (4) the Commission's Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC, and at the local public document room located at the Ocean County Library, Reference Department, 101 Washington Street, Toms River, NJ 08753.

Dated at Rockville, Maryland, this 21st day of February 1995.

For the Nuclear Regulatory Commission.

Alexander W. Dromerick, Sr.

Project Manager, Project Directorate I-4, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

[FR Doc. 95-4977 Filed 2-28-95; 8:45 am]

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[Docket No. 50-483]

Union Electric Company; Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-30, issued to Union Electric Company (the licensee), for operation of the Callaway plant, located in Callaway County, Missouri.

The proposed amendment would modify Technical Specification (TS) Section 3/4.9.1 to establish administrative controls to address a possible boron dilution event directly from the reactor makeup water (RMW) system. An unreviewed safety question was involved with the use of RMW to rinse items removed from the refueling pool and to spray down the refueling pool walls during the pool drain evolution. The use of RMW in prior refueling outages during these Mode 6 activities raised the possibility of a different type of accident than any previously evaluated in the Callaway Final Safety Analysis Report (FSAR).

FSAR Section 15.4.6.2 currently states that administrative controls during Mode 6, i.e., closing and locking dilution source manual valves, preclude an inadvertent dilution of the boron concentration of the primary system. Since these valve closures do not preclude the potential dilution scenario described above, different procedural controls are required to ensure that LCO 3.9.1 boron concentration limit of 2000 ppm is met.

NRC Generic Letter 85-05, "Inadvertent Boron Dilution Events," January 1985, and NSAC-183, "Risk of PWR Inadvertent Criticality During Shutdown and Refueling," dated December 1992, documents the technical justification for determining that boron dilution events are self-limiting. Based on the analyses provided in these documents, the staff's acceptance criteria remains valid for the different boron dilution transient (i.e., that gradual boron dilution events are